High- and Low-Confinement Modes in Simple Magnetized Toroidal Plasmas

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Three-field simulations of interchange turbulence are presented for a simple magnetized toroidal plasma with a vertical magnetic field. The simulations show the presence of two turbulent regimes characterized by low (L) and high (H) confinement properties. We evaluate analytically the properties of the L regime, obtaining expressions for the plasma gradients and for the density and heat fluxes that agree well with the simulations. By increasing the plasma source strength or reducing the vertical magnetic field, a transition to a H regime occurs, in which a strong velocity shear limits the perpendicular transport with respect to the L scaling and the plasma profiles steepen. The analytic estimate of the transition condition is in accord with the simulations.

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We consider plasma turbulence in a simple magnetized torus (SMT) in which a vertical magnetic field B_v , superposed on a toroidal field B_{ϕ} , creates helicoidal field lines with both ends terminating on the torus vessel. As in the scrape-off layer (SOL) of magnetic fusion devices, the turbulence driven by magnetic curvature and plasma gradients causes the plasma to diffuse in the radial direction, while at the same time it is progressively lost due to flows along the field lines. The SMT, employed since the 1980s in the context of tokamak preionization [1], is used for basic plasma physics studies [2,3]. The configuration facilitates the experimental study of low frequency instabilities and the related turbulence and cross-field transport, as it allows more detailed diagnostics and wider parameter scans than are usually possible in major confinement devices. Moreover, in addition to its intrinsic interest, the SMT's relative simplicity provides a useful test bed in which to explore the physics that presumably underlies transport in the SOL of more complex geometries.

In the present Letter, we describe three-field simulations of plasma turbulence in the SMT configuration. For concreteness we focus on the typical parameters of the toroidal plasma experiment (TORPEX) [3], confining an essentially electrostatic, $\beta \ll 1$, plasma with $T_i \ll T_e$. Furthermore, we restrict our attention to the cases in which the interchange-driven transport dominates over drift wave transport, i.e., $D_D/D_I < 1$, where D_D and D_I are the anomalous diffusion coefficients due to drift waves and the interchange instability, respectively. According to mixing length estimates $D_D \sim c_s \rho_s^2/L_p$, where L_p is the pressure scale length, $c_s = \sqrt{T_e/m_i}$, $\rho_s = c_s/\Omega_i$, and $\Omega_i =$ $eB_0/(m_ic)$. Later we show that $D_I \sim (c_s/k_\Delta)\sqrt{2L_p/R}$, where R is the major radius of the torus and $k_{\Delta} = 2\pi/\Delta$ the minimum perpendicular wavelength of the interchange instability, constrained by the return of the field lines in the poloidal plane, i.e., by the distance $\Delta = 2\pi R B_v/B_\phi = L_v/N$ (N is the number of turns of a field line and L_v the SMT height; assuming a high aspect ratio, we take Δ constant). Thus, interchange transport is expected to dominate over drift waves if $k_\Delta \rho_s^2 [R/(2L_p^3)]^{1/2} < 1$.

From drift-reduced Braginskii equations [4], we derive simple two-dimensional fluid equations. We assume that $B_{\nu} \ll B_{\phi}$, so that $B \simeq B_0 R/r$, and note that magnetic curvature is constant along a field line and equal to r. We denote the radial direction with x, the parallel direction with z, and the direction perpendicular to x and z with y. The Braginskii equations are integrated in the parallel direction to evolve the line-integrated density n(x, y) = $\int \mathcal{N}(x, y, z) dz/L_c$, potential $\phi(x, y) = \int \Phi(x, y, z) dz/L_c$, and temperature $T_e(x, y) = \int \mathcal{T}_e(x, y, z) dz/L_c$, $L_c =$ $2\pi NR$ being the magnetic field line length. Reference [5] shows that the ion parallel flow at the sheath edge can be approximated as $\hat{\Gamma}_{\parallel,i} = nc_s/2$, and the electron flow as $\Gamma_{\parallel,e} = nc_s \exp(-e\phi/T_e + \Lambda)/2$, with $\Lambda =$ $\log \sqrt{m_i/(2\pi m_e)}$, where the presheath drops (mainly localized in a small region next to the sheath) in the evaluation of the z integrals defining n and ϕ are neglected. With the Boussinesq approximation for the polarization drift [6], the fluid equations are

$$\frac{\partial n}{\partial t} = \left[\frac{c\phi}{B_0}, n \right] + \frac{2c}{eRB_0} \left(n \frac{\partial T_e}{\partial y} + T_e \frac{\partial n}{\partial y} - en \frac{\partial \phi}{\partial y} \right)
+ D\nabla^2 n - \frac{\sigma n c_s}{R} \exp(\Lambda - e\phi/T_e) + S_n, \tag{1}$$

$$\frac{\partial \nabla^2 \phi}{\partial t} = \left[\frac{c \phi}{B_0}, \nabla^2 \phi \right] + \frac{2B_0}{c m_i R} \left(\frac{T_e}{n} \frac{\partial n}{\partial y} + \frac{\partial T_e}{\partial y} \right) + \nu \nabla^4 \phi + \frac{\sigma c_s m_i \Omega_i^2}{\sigma^2} [1 - \exp(\Lambda - e \phi / T_e)], \tag{2}$$

$$\frac{\partial T_e}{\partial t} = \left[\frac{c \phi}{B_0}, T_e \right] + \frac{4c}{3eRB_0} \left(\frac{7}{2} T_e \frac{\partial T_e}{\partial y} + \frac{T_e^2}{n} \frac{\partial n}{\partial y} - e T_e \frac{\partial \phi}{\partial y} \right)
+ k_e \nabla^2 T_e - \frac{2}{3} \frac{\sigma T_e c_s}{R} \left[1.71 \exp(\Lambda - e \phi / T_e) - 0.71 \right]
+ S_T,$$
(3)

where S_n and S_T represent particle and heat sources, $\sigma = R/L_c = \Delta/(2\pi L_v)$, and $[a, b] = \partial_x a \partial_y b - \partial_y a \partial_x b$. A similar system of equations has been used in Ref. [7]. We denote equilibrium (i.e., time-averaged) quantities with the overbar and fluctuations with a tilde (e.g., $n = \bar{n} + \tilde{n}$).

Relative to the ideal growth rate, the linear growth γ associated with Eqs. (1)–(3) is reduced at low k_y because of sheath effects and at high k_y because of diffusion. Herein, however, we focus on systems in which sheath effects do not affect strongly the linear properties of (1)-(3), i.e., $k_{\Delta}^2 \rho_s^2 \gtrsim \sigma [L_p/(2R)]^{1/2}$. Thus, the longest wavelength mode allowed in the system corresponds to the fastest growing instability, with a growth rate comparable to the interchange growth rate $\gamma \sim \sqrt{2}c_s/\sqrt{RL_p} \equiv \gamma_0$. The nonlinear equations (1)-(3) are solved with a numerical code developed from the ESEL code [8] that implements the algorithm described in Ref. [9]. We consider a rectangular domain with extension L_x and Δ along x and y, respectively. Dirichelet boundary conditions are imposed at x = $0, L_{\rm r}$, and, because of the flute character of the interchange mode, periodic boundary conditions are assumed along y. The simulations are started with small amplitude random noise in n, ϕ , and T_e . The sources then introduce plasma and heat, increasing the plasma pressure and triggering the interchange instability. After a transient phase, a quasistationary state is reached in which the plasma, generated by the source and transported by the interchange dynamics in the radial direction, is eventually removed from the system by parallel losses. The results described here focus on this quasistationary state and its convergence has been checked by following simulations up to $t\gamma_0 \simeq 4000$.

We normalize n and T_e to reference values n_0 and T_{e0} , time t to R/c_{s0} ($c_{s0}=\sqrt{T_{e0}/m_i}$), and lengths to $\rho_{s0}=c_{s0}/\Omega_i$. We use R=200, $L_v=64$, $L_x=200$, D=1, v=0.5, $k_e=1$, and $\Lambda=3$. The sources are defined as $S_n=S_T=S_0\{S_{\rm UH}\exp[-(x-x_{\rm UH})^2/\lambda_{\rm UH}^2]+S_{\rm EC}\exp[-(x-x_{\rm EC})^2/\lambda_{\rm EC}^2]\}$ ($S_{\rm UH}=1.5$, $S_{\rm EC}=1$, $\lambda_{\rm UH}=5$, $\lambda_{\rm EC}=2.5$, $x_{\rm UH}=35$, $x_{\rm EC}=15$), in order to mimic the upper hybrid (UH) and electron cyclotron (EC) sources in TORPEX [10]. The simulation results are insensitive to the details of the plasma source. We perform a set of simulations in which we vary σ and S_0 ($\sigma=0.03$, 0.04, 0.05, 0.07, 0.08, $S_0=0.01$, 0.025, 0.05, 0.1, 0.25, 0.5, 1, 2, 4, 8).

The simulations show the presence of two turbulent regimes. Typical snapshots of ϕ from a simulation in the first regime, denoted as the low-confinement mode (L mode), are shown in the upper panels in Fig. 1. In this regime, background shear flow effects are negligible, and the interchange instability displays an intermittent character, with the emission of plasma blobs that transfer plasma

from the source region to the low-field region while the plasma is simultaneously drained by parallel absorption. The profiles of \bar{n} , \bar{T}_e , and $\bar{\phi}$ show an exponential decay on the low-field side where the interchange dynamics is active (i.e., for $x > x_S$, x_S denoting the right-most peak in density and temperature). By increasing S_0 or reducing σ , a new turbulent regime is reached, denoted as the H mode, in which a strong $\mathbf{E} \times \mathbf{B}$ shear flow appears. This flow limits the perpendicular diffusion coefficient, thereby causing a reduction of the flux with respect to the L-mode estimate and steeper plasma profiles. The lower panels in Fig. 1 show typical snapshots and profiles in the H-mode regime.

We first consider turbulence in the L-mode regime. Let us estimate the y-averaged cross-field particle transport $\Gamma_n = R \langle \tilde{n} \partial_{\nu} \tilde{\phi} \rangle_{\nu}$. We first note that in the quasistationary state, since $L_n \le R$ and $\sigma \le 1$, the dominant terms in Eq. (1) lead to $\partial_t n + R[\phi, n] \simeq 0$. Since in the L mode the shear flow plays a negligible role, one has $\partial_{\nu} \tilde{\phi} \sim$ $\gamma_0 \tilde{n}/(R \partial_x \bar{n})$, which is well satisfied by simulation data. Turbulence saturation occurs when the instability drive is removed from the system, i.e., when $\partial_x \bar{n} \sim \partial_x \tilde{n}$. By following standard local theory methods (outlined, e.g., in Ref. [11] for drift waves), it is possible to estimate the typical radial extension of the interchange mode in the main regime of interest here: $k_y L_p > 1$. The leading order (i.e., ideal MHD interchange) terms in the system (1)–(3) reduce to $\partial_x^2 \tilde{\phi} - k_y^2 [1 + G(x)] \tilde{\phi} = 0$, where G(x) = $-2R\partial_x p_e/(n\gamma^2)$. By Taylor-expanding $G(x) \simeq G_0 +$ $G_0''(x-x_0)^2/2$, with x_0 the point of instantaneous steepest pressure gradient, $G_0 = G(x_0)$, and $G_0'' = \partial_x^2 G(x)|_{x=x_0} \sim$ $2RT_e/(\gamma^2 L_p^3)$, one is lead to a harmonic oscillator equation for $\tilde{\phi}$. In the limit $k_y L_p > 1$, one has $\gamma = \gamma_0|_{x=x_0}$ and $\tilde{\phi} \propto$ $\exp[-a(x-x_0)^2/2]$, with $a = k_v(|G_0''|/2)^{1/2} \sim k_v/L_p$. The typical radial extension of the interchange mode is

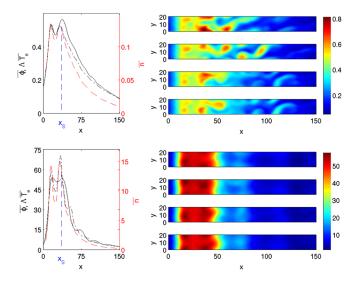


FIG. 1 (color online). $\bar{\phi}$ (solid black curve), \bar{T}_e (dashed-dotted black curve), and \bar{n} (dashed red curve) and typical ϕ snapshots for $S_0=0.01$ (upper panels) and $S_0=8$ (lower panels). $\sigma=0.05$.

thus given by $1/\sqrt{a} \sim (L_p/k_v)^{1/2}$, and, by considering the dominant mode $k_y = k_\Delta$, one estimates $\partial_x \tilde{n} \sim \tilde{n} (k_\Delta / L_p)^{1/2}$, which finally gives $\tilde{n} \sim \bar{n} (L_p/k_\Delta)^{1/2}/L_n$ (comparison with simulation data shows that the latter overestimates the density fluctuations by a factor of ≈ 2). The evaluation of \tilde{n} and $\partial_{\nu}\tilde{\phi}$ leads to estimates of the radial density flux in the *L*-mode regime as $\Gamma_{n,L} = \bar{n}(2RL_p\bar{T}_e)^{1/2}/(L_nk_\Delta)$ and to an expression for the temperature flux $\Gamma_{T,L}$ = $(2RL_p\bar{T}_e^3)^{1/2}/(L_Tk_\Delta)$ [or equivalently, a diffusion coefficient $D_I = (2RL_p\bar{T}_e)^{1/2}/k_{\Delta}$]. We note that the Kelvin-Helmholtz (KH) instability could provide an alternative saturation mechanism, but numerical tests (discussed later) show that KH instability does not play a significant role in the L mode. In Fig. 2, we compare the values of Γ_n and Γ_T from the simulations with the L-mode estimates, both quantities being evaluated at $x = x_S$, with $L_n(L_T)$ deduced from the radial distance between n_{max} and $n_{\text{max}}/2$ ($T_{e,\text{max}}$ and $T_{e,\text{max}}/2$). Two sets of simulations are shown: In the first set, corresponding to all values of S_0 for the highest σ and small values of S_0 for small σ , both Γ_n and Γ_T follow the predicted scaling laws. These simulations belong to the L-mode regime. In the second set, i.e., small σ and high S_0 , the transport level is smaller than the predictions. Figure 3(a) shows that the discrepancies with the L-mode predictions are related to shear flow stabilization. The H factor, defined as $H_n = \Gamma_{n,L}/\Gamma_n$ and $H_T = \Gamma_{T,L}/\Gamma_T$, is plotted as a function of $\gamma_0/v'_{\mathbf{E}\times\mathbf{B}}$, where γ_0 is evaluated at $x = x_S$, while $v'_{\mathbf{E} \times \mathbf{B}} = \partial_x^2 \phi$ denotes the maximum shear flow on the low-field side (H_n and H_T vary by $\sim 15\%$ along x). For $\gamma_0/v'_{E\times B} > 0.7$ the simulations fall in the L-mode regime, where the previously obtained scaling is obeyed: $\Gamma_n \simeq \alpha \Gamma_{n,L}$ and $\Gamma_T \simeq \alpha \Gamma_{T,L}$, with $\alpha = 0.3 \pm 0.03$. The confinement improves when the shear flow increases: We define the L-H transition to occur when $\Gamma_n \simeq \alpha \Gamma_{n,L}/3$ and $\Gamma_T \simeq \alpha \Gamma_{T,L}/3$; this takes place for $\gamma_0/v'_{\rm E \times B} \simeq 0.5$ (vertical purple lines in Fig. 3).

We now show that, in the L-mode regime, L_n and L_T depend only on the geometric parameters of the SMT. As illustrated in Fig. 1, the potential is constrained to follow $\phi \simeq \Lambda T_e$ due to the exponential dependence of $\Lambda - \phi/T_e$ in Eqs. (1)–(3). By integrating Eqs. (1) and (2) along y, time averaging, and neglecting diffusivity, one obtains

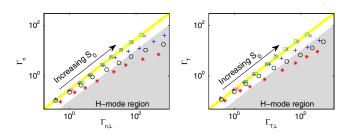


FIG. 2 (color online). Γ_n and Γ_T vs $\Gamma_{n,L}$ and $\Gamma_{T,L}$, for $\sigma = 0.03$ (red stars), $\sigma = 0.04$ (black circles), $\sigma = 0.05$ (blue crosses), $\sigma = 0.07$ (magenta \times), and $\sigma = 0.08$ (green squares). The shaded area is the H-mode region.

$$\frac{\partial \Gamma_n}{\partial x} \simeq S_n - \sigma \bar{n} \bar{T}_e^{1/2}, \qquad \frac{\partial \Gamma_T}{\partial x} \simeq S_T - \frac{2}{3} \sigma \bar{T}_e^{3/2}.$$
 (4)

Within the L regime, Eqs. (4) admit solutions of the form $\bar{n}=n_{\max}\exp[(x-x_S)/L_{n,L}]$ and $\bar{T}_e=T_{e,\max}\exp[(x-x_S)/L_{T,L}]$ on the low-field (right) side of the source free region, i.e., for $x>x_S$ (where $x_S\simeq 36$ in the present simulations). By substituting the expression of $\Gamma_{n,L}$ and $\Gamma_{T,L}$ for Γ_n and Γ_T in Eqs. (4), we obtain $\alpha\sqrt{2L_{p,L}}/L_{n,L}[1/L_{n,L}+1/(2L_{T,L})]=1/(L_v\sqrt{R})$ and $\alpha\sqrt{2L_{p,L}}/L_{T,L}^2=4/(9L_v\sqrt{R})$, thus finding $L_{n,L}\simeq 0.58(L_v\sqrt{R})^{2/3}$ and $L_{T,L}\simeq 0.74(L_v\sqrt{R})^{2/3}$, which, as claimed, depend only on the geometry of the machine. In Fig. 3(b), we plot $L_{T,L}/L_T$ vs $\gamma_0/v_{E\times B}'$: As expected, in the case of negligible shear flow, L_n and L_T do not depend on S_0 , σ , and x (within 15% oscillation); in the H mode, the reduction of the diffusion coefficient with respect to D_I leads to smaller L_n and L_T .

We now turn to the L-H transition threshold. Insight into the nature of the velocity shear is given by the time-averaged and y-integrated Eq. (2), $R\partial_x^2 \langle \tau_{xy} \rangle_y =$ $\sigma \langle \sqrt{T_e[1 - e^{(\Lambda - \phi/T_e)}]} \rangle_v$, where $\tau_{xy} = \overline{\partial_x \tilde{\phi} \partial_y \tilde{\phi}}$ is the xy component of the turbulence-induced Reynold stress (RS), having neglected $\nu \nabla^4 \phi$ and approximated $\langle \overline{T_e \partial_\nu \log n} \rangle_\nu \simeq$ 0. Since deviations of the potential from $\phi = \Lambda T_e$ are due to the presence of the RS, shear flow can be related either to a sheared temperature profile (STP), i.e., $\partial_x^2 \phi \simeq \Lambda \partial_x^2 T_e$, or to RS-induced shear. In both cases, the amplitude of the shear flow is limited by the KH instability. Reference [12] evaluates the KH growth rate in the case of a Bickley jet and shows that KH is stable if $k_{\nu}L_{\phi} > \zeta$ ($\zeta \simeq 2$). Analogously, in our system, the KH instability imposes $k_{\nu}L_{\phi} > \delta$, where the most stringent criterion for stability is obtained for $k_y = k_\Delta$, i.e., $L_\phi > \delta \Delta/(2\pi)$. In most of our simulations, shear flow tends to reach the maximum value allowed by the KH instability, i.e., $L_{\phi} \simeq \delta \Delta/(2\pi)$, so that, in general, $v'_{E\times B} \simeq \chi \phi_{max}/\Delta^2$, with $\chi \simeq 2.5$ ($\delta = 2\pi/\sqrt{\chi}$).

In the L regime, one observes that $\partial_x^2 \bar{\phi} > \Lambda \partial_x^2 \bar{T}_e$ and thus that shear flow is mostly related to the RS term. However, L-mode simulations are not significantly affected when the y-averaged RS term is artificially removed, thus showing that shear flow plays no significant role in this regime. When approaching the H mode, the

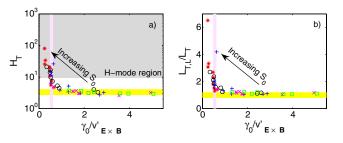


FIG. 3 (color online). H_T and $L_{T,L}/L_T$ vs $\gamma_0/v_{E\times B}'$; same notation as Fig. 2. The plots of H_n and $L_{n,L}/L_n$ (not shown) reveal essentially identical dependences.

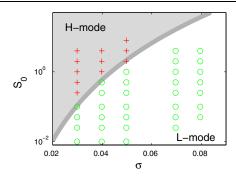


FIG. 4 (color online). L-H threshold from Eq. (5) and simulations falling in the H-mode (red crosses) and L-mode (green circles) regimes.

shear flow amplitude increases, and the relative importance of the contribution due to the STP is enhanced. At the *L-H* threshold, one then observes a comparable contribution to the shear flow from the RS and the STP term, and thus $\partial_x^2 \phi \sim \Lambda \partial_x^2 T_e$, i.e., $L_T \sim L_\phi$.

Assuming that $L_n \sim L_T$ and $\phi_{\rm max} \sim \Lambda T_{e,{\rm max}}$ leads to $\gamma_0/v'_{\rm E\times B} \simeq 2\Delta^{3/2}/(\chi\Lambda\sqrt{RT_{e,{\rm max}}})$ at the *L-H* threshold. The value $T_{e,{\rm max}}$ can be deduced by integrating Eq. (4) along the *x* direction across the entire domain. An exponential dependence of the temperature is assumed for $x > x_S$ and, to take into account the diffusion-dominated dynamics for $x < x_S$, a shape factor ξ is introduced such that $\int_0^{x_S} T_e^{3/2} dx = \xi T_{e,{\rm max}}^{3/2} x_S$ ($\xi \simeq 0.5$). One concludes that $T_{e,{\rm max}}^{3/2} \simeq \int S_T dx/[\sigma(2\xi x_S/3 + 4L_T/9)]$. The *L-H* transition occurs for $\gamma_0/v'_{\rm E\times B} \simeq 0.5$, which leads to

$$\frac{2\Delta^{11/6}}{\chi \Lambda \sqrt{R}} \left(\frac{4/9\Delta + 2\xi x_S/3}{2\pi L_v \int S_T dx} \right)^{1/3} \simeq 0.5.$$
 (5)

Equation (5) can be used to find the relation between S_0 and Δ at the *L-H* threshold. With the simulation parameters, the threshold condition can be written as $S_0 = 8.54 \times 10^{-8} \Delta^{11/2} (1 + 0.039 \Delta)$. In Fig. 4, the *L-H* threshold prediction shows good agreement with the simulations.

Deep into the H mode, the relative contribution to the shear flow from the RS and the STP remain comparable. In this regime the RS is essential. Contrary to the L-mode simulations, if the y-averaged RS term is artificially removed from the model, the H-mode simulations are deeply affected. One observes the development of radial streamers that continuously transport plasma from the source region to the right boundary of the domain, degrading the perpendicular confinement. This is related to the fact that the RS is essential for the formation of sheared flows and the breakup of radial streamers via, for example, the KH instability. We also point out that, in the H mode, the regions within the barrier in which the temperature profile flattens, $\partial_x \bar{T}_e \simeq 0$, are centered on the location where velocity shear is minimum, $\partial_x^2 \bar{\phi} \simeq 0$, i.e., where the interchange mode is not stabilized.

We finally note that the energy confinement time can be estimated as $\tau(L_T) = \int T_e dx / \int S_T dx \simeq (L_T + \xi x_S) /$

 $[\sigma(\int S_T dx)^{1/3}(2\xi x_S/3 + 4L_T/9)^{2/3}]$, with the *L*-scaling prediction being $\tau_L = \tau(L_{T,L})$. If one attempts to evaluate the gain in the confinement time, one observes that $\tau/\tau_L \leq 1$ for $L_T/L_{T,L} \leq 1$. Contrary to the tokamak case, the *H*-mode confinement time thus decreases with respect to the *L* scaling. This results from the fact that in a SMT all field lines are subject to higher parallel sheath losses as T_e increases in the *H* mode.

The intermittent turbulence recently reported in TORPEX [13] corresponds, according to the theory outlined here, to the L-mode regime. The estimate of D_D/D_L shows that the drift wave instability becomes more important for small B_{ν} or steep plasma profiles, which are the conditions typical for the H regime to appear. Fully threedimensional simulations are required to study the effect of the drift wave dynamics on the system and how it is impacted by the sheared flows discussed here. Strong similarities exist between the tokamak and the SMT L-H transition. In fact, as in the tokamak, the H mode is driven by the increase of the input power and is characterized by strong shear flows leading to a steeper density and temperature gradients, higher peak density and temperature, and reduced radial diffusion coefficients. However, contrary to the tokamak case, the confinement time is reduced in the SMT, as a result of increased sheath losses.

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- [1] S. Nakao et al., Phys. Lett. 96A, 405 (1983).
- [2] E. D. Zimmerman and S. C. Luckhardt, J. Fusion Energy 12, 289 (1993); K. Rypdal and S. Ratynskaia, Phys. Rev. Lett. 94, 225002 (2005); P. K. Sharma and D. Bora, Plasma Phys. Controlled Fusion 37, 1003 (1995); J. C. Perez et al., Phys. Plasmas 13, 032101 (2006); C. Riccardi et al., Plasma Phys. Controlled Fusion 36, 1791 (1994).
- [3] A. Fasoli et al., Phys. Plasmas 13, 055902 (2006).
- [4] A. Zeiler et al., Phys. Plasmas 4, 2134 (1997).
- [5] P. C. Stangeby, *The Plasma Boundary of Magnetic Fusion Devices* (Institute of Physics, Berkshire, 2000).
- [6] C. Q. Yu et al., Phys. Plasmas 13, 042508 (2006).
- [7] N. Bisai et al., Phys. Plasmas 11, 4018 (2004).
- [8] V. Naulin et al., Phys. Rev. Lett. 81, 4148 (1998); O. E. Garcia et al., Phys. Rev. Lett. 92, 165003 (2004).
- [9] V. Naulin and A. H. Nielsen, SIAM J. Sci. Comput. 25, 104 (2003).
- [10] M. Podestà *et al.*, Plasma Phys. Controlled Fusion **48**, 1053 (2006).
- [11] B. N. Rogers and W. Dorland, Phys. Plasmas 12, 062511 (2005).
- [12] D. Biskamp, *Magnetic Reconnection in Plasma* (Cambridge University Press, Cambridge, England, 2000).
- [13] I. Furno et al., Phys. Rev. Lett. 100, 055004 (2008).